

addition to that in the tumour with commercial gallium-67 is however its serious drawback [2]. We have succeeded in avoiding this inconvenience by injecting gallium-67 made free just before its administration [3]. In this way we could image malignancies even in the liver which has not been reported earlier due to high concentration of the radionuclide in the healthy organ [1–3]. But, the preparation of free or loosely bound gallium-67 from commercial gallium-67 citrate is not easy because the composition of these solutions is not always the same [4]. Literature reports the administration of iron or iron complexes for improving the quality of tumour image with gallium-67 [5]. Although a more rapid tumour to blood radioactivity ratio is obtained by the administration of these substances, they invariably lead to reduction in the radionuclide uptake in the tumour and thus to low quality image. We have examined the effect of administration of low doses of lanthanum on the rate of elimination and on the tumour affinity for gallium-67.

Aqueous solutions of different salts of lanthanum(III) were administered in Morris hepatoma-3924A-bearing rats after the injection of commercial gallium-67 citrate solutions. Whole body scintigram of the rat was taken at different intervals of time after the administration of the radionuclide. After the animal died, the radioactivity in different organs was measured with a well-type gamma counter.

Lanthanum administration although favours the elimination of the radioactivity from the blood it does not improve the quality of the tumour image. The distribution of gallium-67 in lanthanum(III)-treated animals and those which did not receive lanthanum will be discussed.

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E17

Separation and Sequential Determination of Americium and Plutonium in Urine Samples

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Simultaneous determination of low levels of Americium and Plutonium in complex matrices has

always been difficult to carry out owing to the chemical behavior differences of the two actinides.

An analytical extraction and separation technique of the two actinides has been realized utilizing Dibutyl, N,N-diethyl carbamylphosphonate (DDCP) as a reagent. The possibility has been ascertained of performing a separation through partition chromatography techniques and through liquid–liquid extraction. In particular, such technique has been applied to the determination of the two actinides in urine samples from personnel exposed to the risk of internal contaminations due to emitter alpha radionuclides. In this case the liquid–liquid extraction technique has been adopted for reasons connected with the operation rapidity and ease.

This method involves: wet way sample mineralization by HNO_3 and evaporation concentration; liquid–liquid extraction of interesting actinides by DDCP from a 12 M solution in HNO_3 ; sequential stripping of Am by HNO_3 2 M and of Pu by HCl 3 M and HN_4I 0.1 M; electrodeposition on a steel plate and final measurement through alpha spectrometry with surface barrier detector.

Average final yields obtained through this analytical method have been: 80% for Americium, 66% for Plutonium. Activity values found in 'urine whites' are comparable with the full-scale value. Min. detectable values at a 300 min count time have been the following: Americium 1.3×10^{-3} Bq/l (0.03 pCi/l), Plutonium 1.6×10^{-3} Bq/l (0.04 pCi/l). At present, experiments are carried out for applying this analytical method on other matrices as well.

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Actinide Behavior and Radiation Damage Produced by α -Decay in Materials to Solidify Nuclear Waste

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Solidification of High Level (radioactive) Waste, HLW, from reprocessing of nuclear reactor fuels in a suitable matrix and subsequent burying in a suitable repository are the envisaged final steps of the nuclear fuel cycle. Because of the long half-lives of some of the actinides (mainly Np, Pu, Am and Cm) and because of their decay by emission of an α -particle (~ 5 MeV energy) and a heavy recoil atom (e.g. U from Pu-decay, ~ 100 keV energy), the actinide behavior in the candidate materials for waste solidification deserves particular attention. The materials for solidification are glasses or certain minerals, or a